



U.S. DEPARTMENT OF
ENERGY

Nuclear Energy

NEET FY 12 Project

**Accelerated Development of Zr-Containing
New Generation Ferritic Steels
for Advanced Nuclear Reactors**

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DOE-NE Materials Crosscut Coordination Meeting

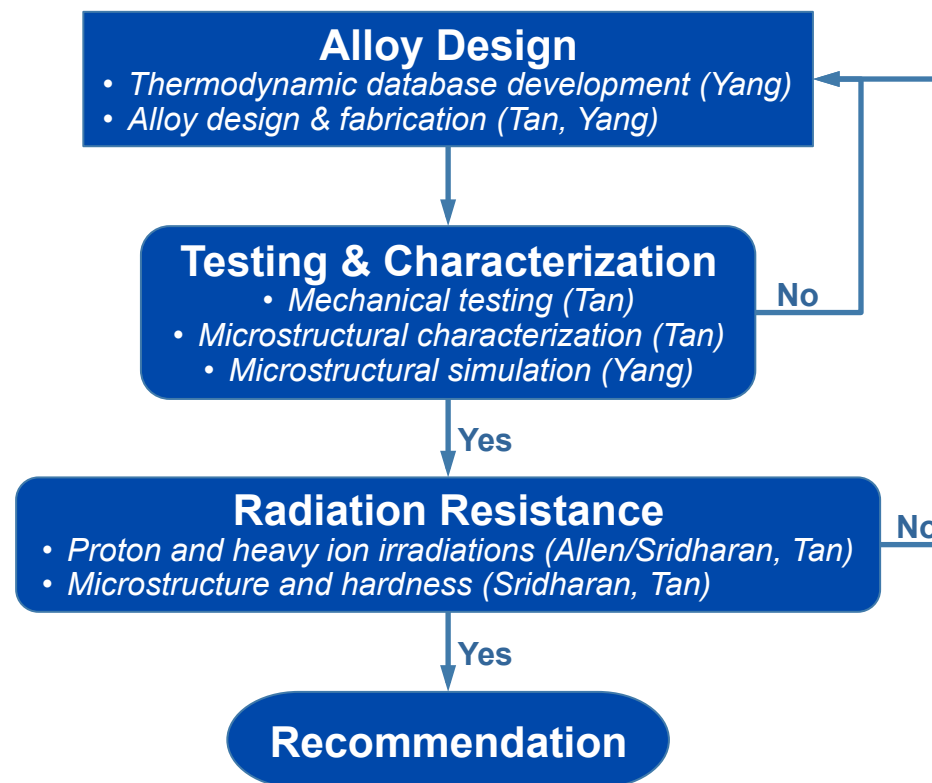
August 20, 2013

Tasks and Team Members

■ Team Members

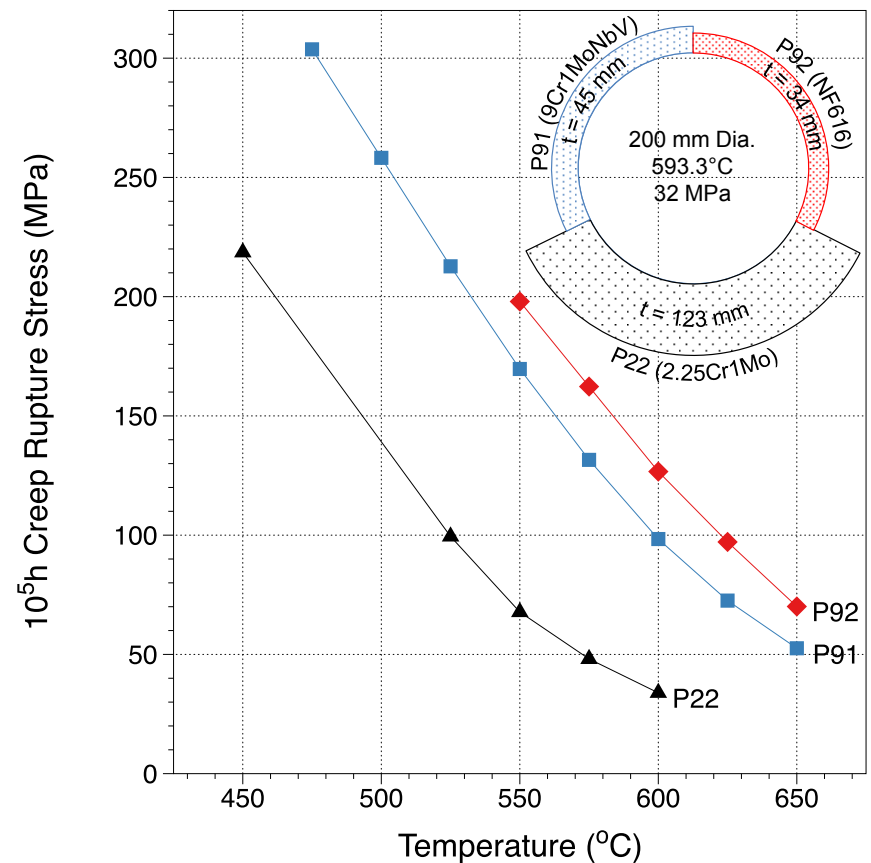
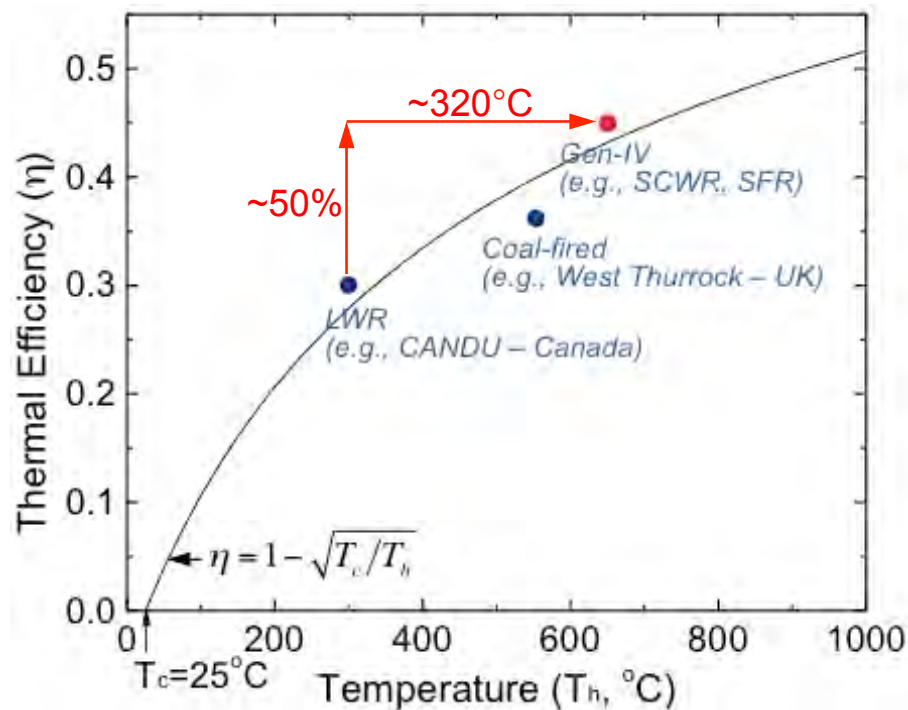
- Lizhen Tan and Ying Yang (ORNL)
- Todd Allen and Kumar Sridharan (UW-Madison)

■ Tasks



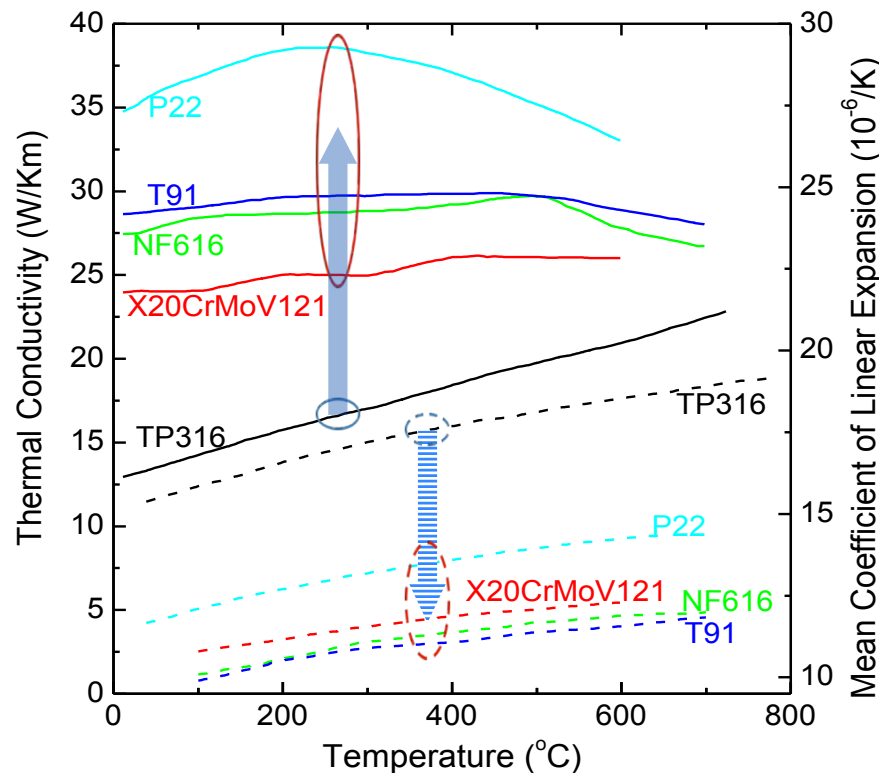
The Need for Advanced Material Development

- The escalating global clean-energy need drives higher operating temperatures of power plants for improved thermal efficiency.
- Advanced materials with superior high-temperature strength can effectively improve plant economics (*reduced commodities, increased thermal efficiency, longer lifetimes*), safety margins, and design flexibility.

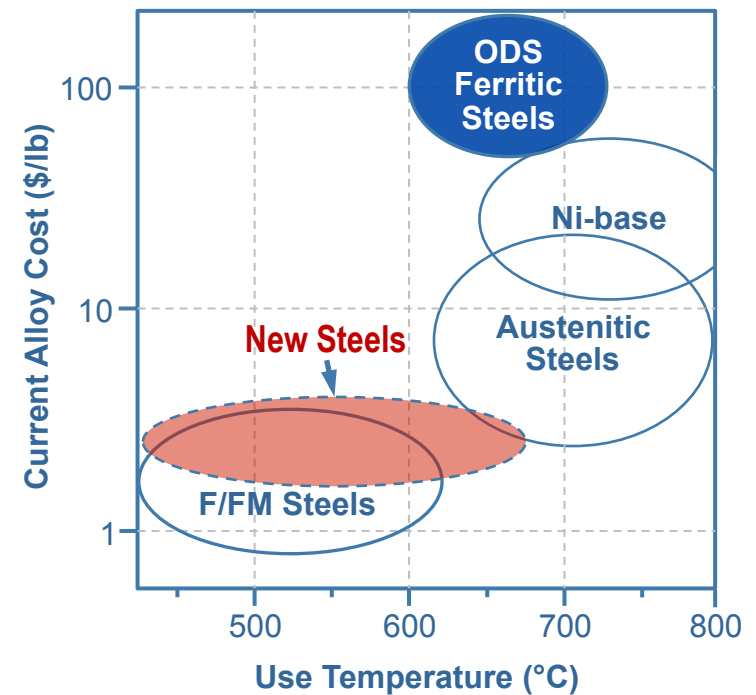


Ferritic Steels Have Outstanding Properties for Engineering Design

- **Ferritic steels are important structural materials for nuclear reactors**
 - Advantages of FM steels over austenitic stainless steels
 - High resistance to radiation-induced void swelling (e.g., ~10 times better at temperatures above 300°C)
 - High thermal conductivity and low thermal expansion



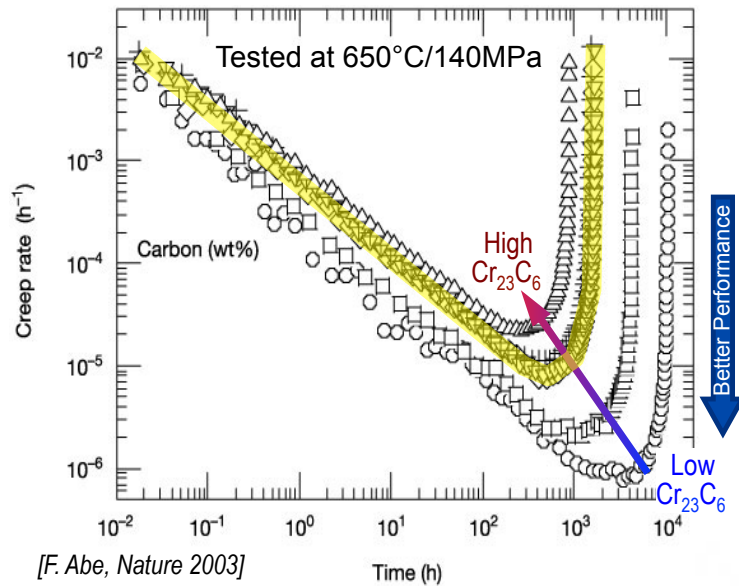
- Low-cost



[Source: adapted from B.A. Pint]

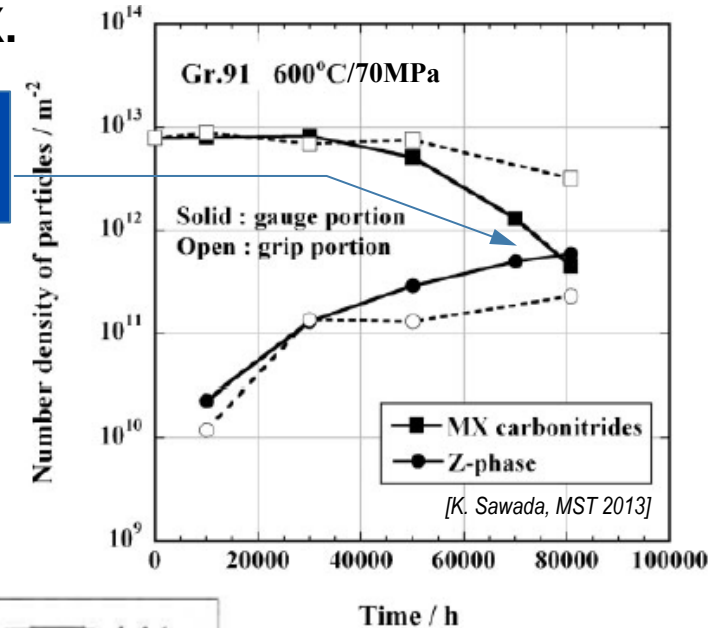
Concerns of The Current FM Steels

- Higher $Cr_{23}C_6$ amount results in greater creep rate.

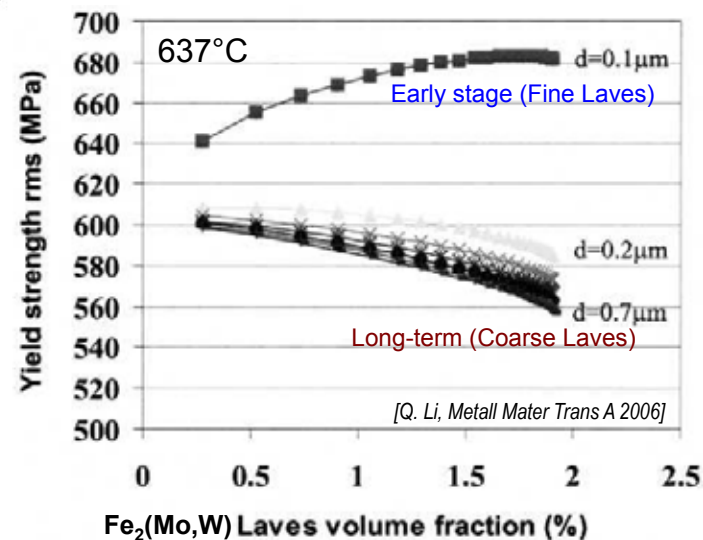


- Coarse Z-phase forms by consuming fine MX.

Stress accelerates the replacement of MX by Z-phase.



- Laves phase coarsening deteriorates strength.



Potential Effects of Zr Alloying

- Refine grain size by ZrC → Improve strength, toughness, etc.
- Reduce the amount of Cr_{23}C_6 → Improve creep resistance, reduce sensitization, etc.
- Reduce radiation-induced segregation (RIS) → Reduce potential SCC/IASCC.
- Form Zr-bearing Laves phase → Unclear.
- Form new phases, e.g., $\text{Fe}_{23}\text{Zr}_6$ → Unclear.

- **Difficulty in alloy impurity control (oxygen): contrary effects**
 - Low enough oxygen → better fracture toughness
 - High oxygen → poor fracture toughness, ductility, etc.

Approach to Accelerate The Development of Zr-Containing Ferritic Steels

Past: Trial and Error Method; Time-consuming and expensive

Experiment

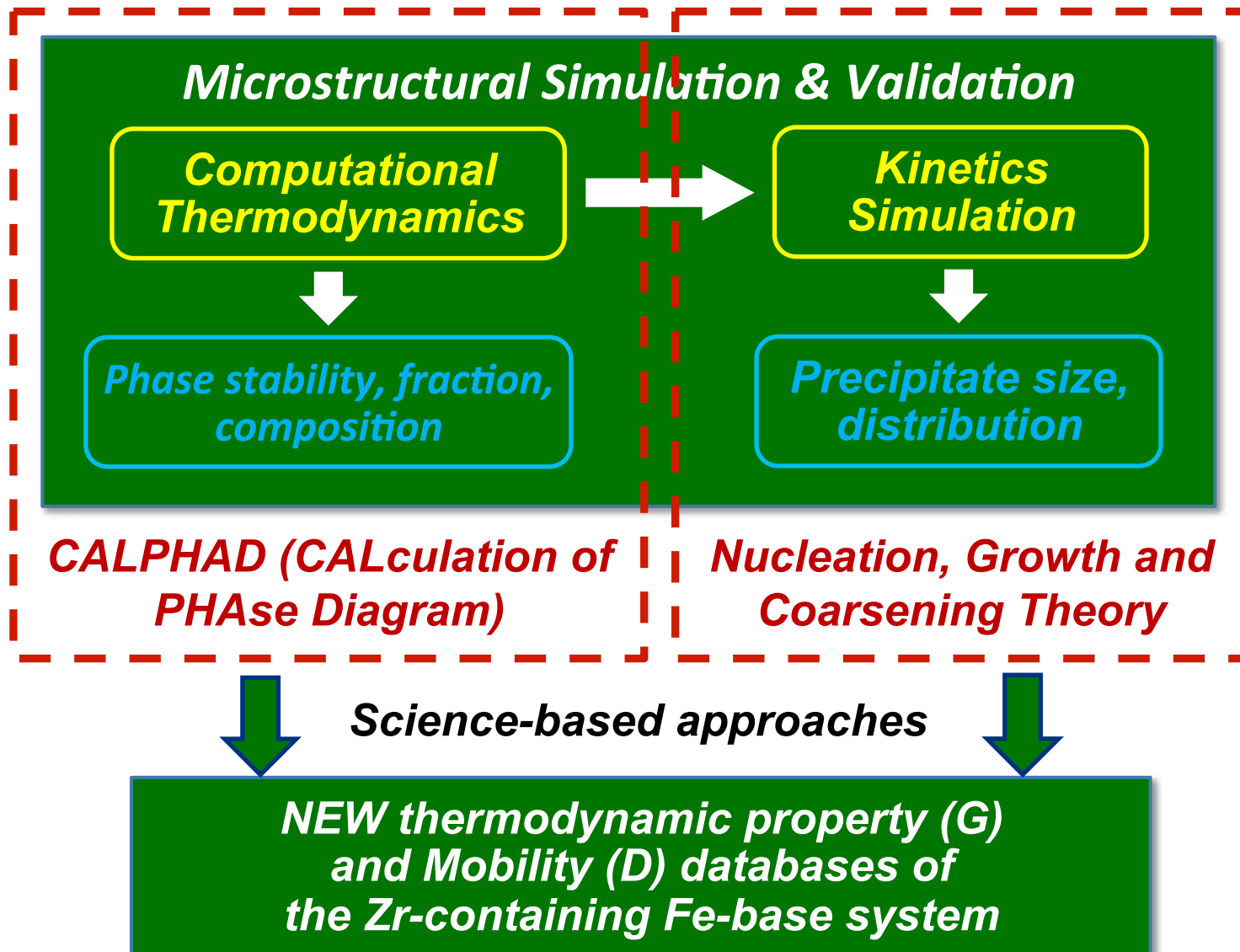
Microstructure

Property

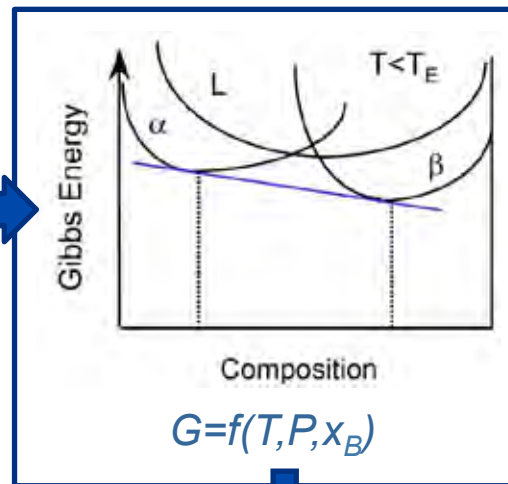
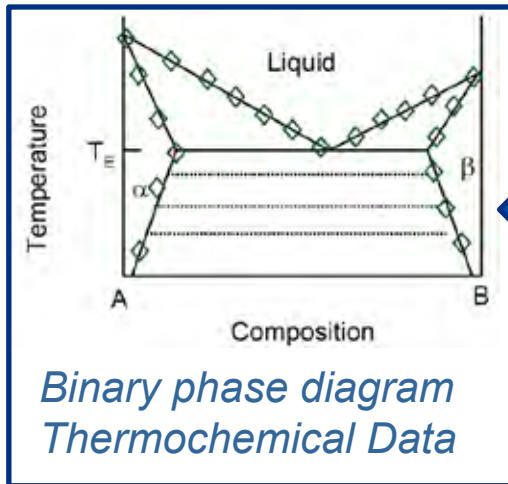
Now/Future: Materials-by-Design; High efficient and low-cost

Computational Tools
(Software + Database)

Computational Microstructural Modeling

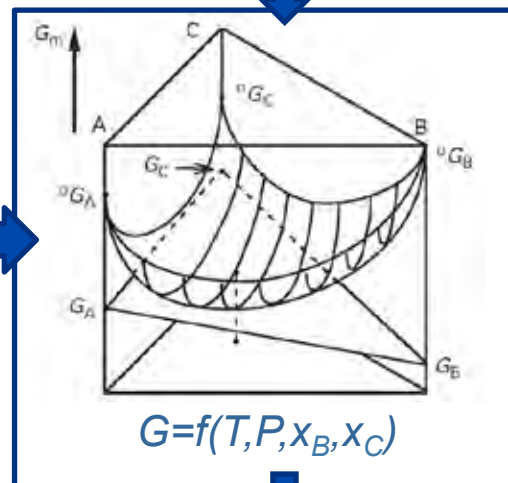
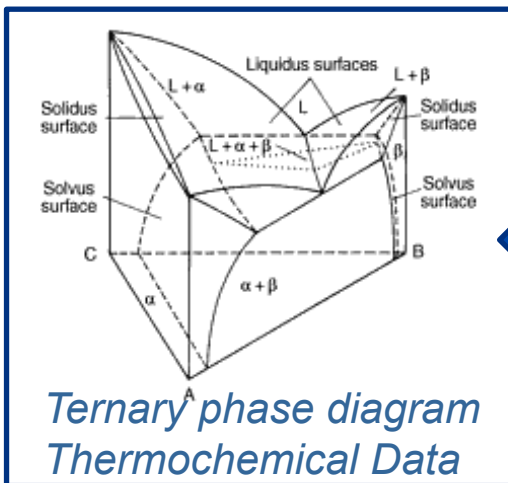


Multicomponent Thermodynamics (CALulation of PHase Diagram-CALPHAD)



$$G = \sum_i X_i G_i^0 + RT \sum_i X_i \ln X_i$$

+ Binary $G^{\wedge}ex$



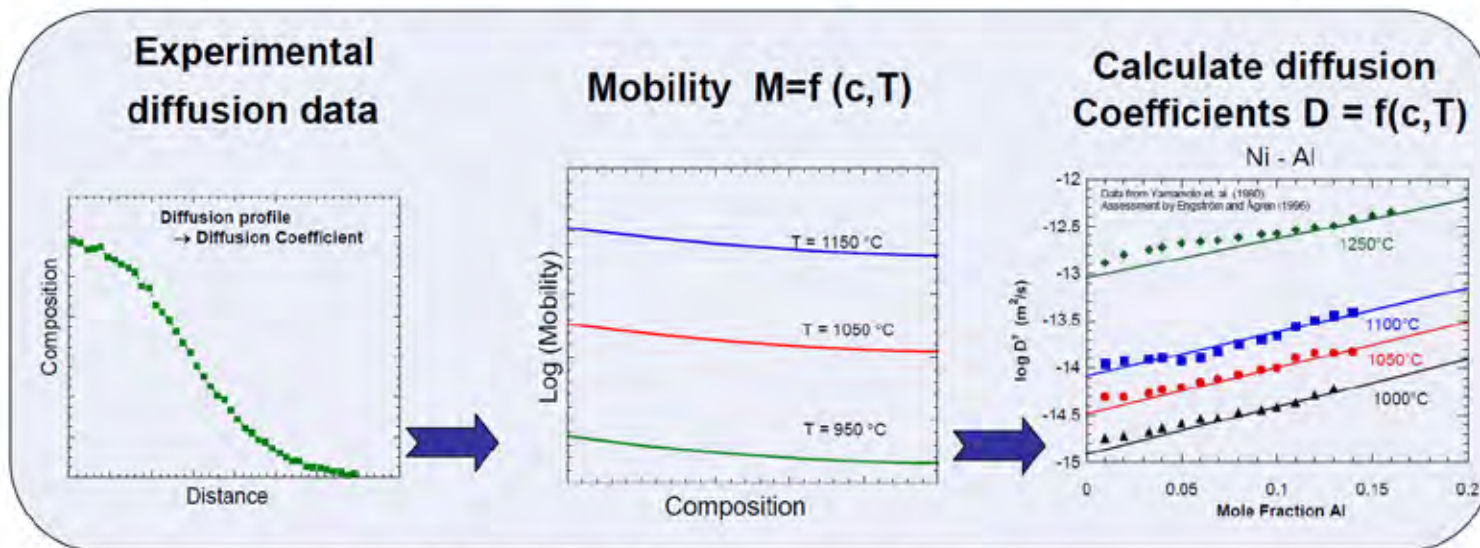
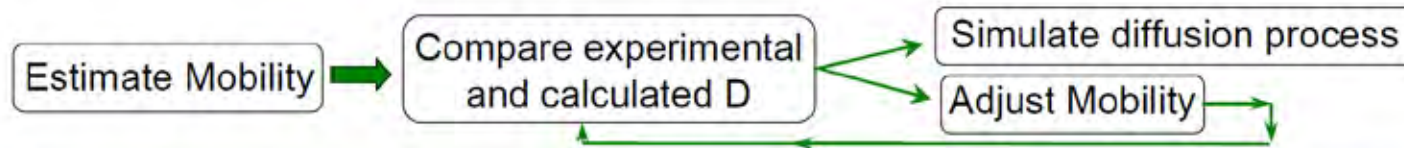
+ Binary $G^{\wedge}ex$ + Ternary $G^{\wedge}ex$

Phase diagram

Thermodynamics

*+ Binary $G^{\wedge}ex$ + Ternary $G^{\wedge}ex$
+ nth order $G^{\wedge}ex$*

Computational diffusivity



$$M_i = \frac{M_i^\circ}{RT} \exp\left(\frac{-\Delta Q_i}{RT}\right) \text{ where } \Delta Q_i = f(c_i, T) \text{ and } M_i^\circ = 1$$

For a binary: $Q_i^\phi = c_i Q_i^i + c_j Q_j^j + c_i c_j (A_i^{i,j} + (c_i - c_j) B_i^{i,j} + (c_i - c_j)^2 C_i^{i,j} + \dots)$

NIST

$$D_{i,k}^* = RT M_k \left(\frac{D_{i,k}^*}{k} \right) \left(\frac{D_{i,k}^*}{k} \right) ; \left(\frac{D_{i,k}^*}{k} \right) = 1 + \frac{\partial \ln(f_{i,k})}{\partial \ln(X_{i,k})} ; \left(\frac{D_{i,k}^*}{k} \right) = \frac{A_i B_i + x A_i B_i}{x B_i D_i + x A_i B_i}$$

Computational tools used in this study

Software

Computational thermodynamics

- *Matcalc 5.51*
- *Pandat 8.0*

Precipitation kinetics

- *Matcalc 5.51*

Database

Thermodynamic property

- *OCTANT (in-house)*

Mobility

- *MCFe (Non-encrypt)*

OCTANT: ORNL Computational Thermodynamics for Applied Nuclear Technology



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Thermodynamic database Fe-C-Cr-Mo-Nb-Ti-W-Zr

Binaries

	C	Cr	Mo	Nb	Ti	W	Zr
Fe	Fe-C	Fe-Cr	Fe-Mo	Fe-Nb	Fe-Ti	Fe-W	Fe-Zr
C		C-Cr	C-Mo	C-Nb	C-Ti	C-W	C-Zr
Cr			Cr-Mo	Cr-Nb	Cr-Ti	Cr-Ti	Cr-Zr
Mo				Mo-Nb	Mo-Ti	Mo-W	Mo-Zr
Nb					Nb-Ti	Nb-W	Nb-Zr
Ti						Ti-W	Ti-Zr
W							W-Zr

Fe-X-Y Ternaries

	Cr	Mo	Nb	Ti	W	Zr
Fe-C	Fe-C-Cr	Fe-C-Mo	Fe-C-Nb	Fe-C-Ti	Fe-C-W	Fe-C-Zr
Fe-Cr		Fe-Cr-Mo	Fe-Cr-Nb	Fe-Cr-Ti	Fe-Cr-W	Fe-Cr-Zr
Fe-Mo			Fe-Mo-Nb	Fe-Mo-Ti	Fe-Mo-W	Fe-Mo-Zr
Fe-Nb				Fe-Nb-Ti	Fe-Nb-W	Fe-Nb-Zr
Fe-Ti					Fe-Ti-W	Fe-Ti-Zr
Fe-W						Fe-W-Zr

X-Y-C Ternaries

	Mo	Nb	Ti	W	Zr
Cr-C	Cr-Mo-C	Cr-Nb-C	Cr-Ti-C	Cr-W-C	Cr-Zr-C
Mo-C		Mo-Nb-C	Mo-Ti-C	Mo-W-C	Mo-Zr-C
Nb-C			Nb-Ti-C	Nb-W-C	Nb-Zr-C
Ti-C				W-Ti-C	Ti-Zr-C
W-C					W-Zr-C

From literature
 From this work

■ Class I: Intermetallics-strengthened alloys

- Support fundamental Zr-bearing thermodynamic database development
- Explore Zr-bearing Laves phase and potential $\text{Fe}_{23}\text{Zr}_6$ effects

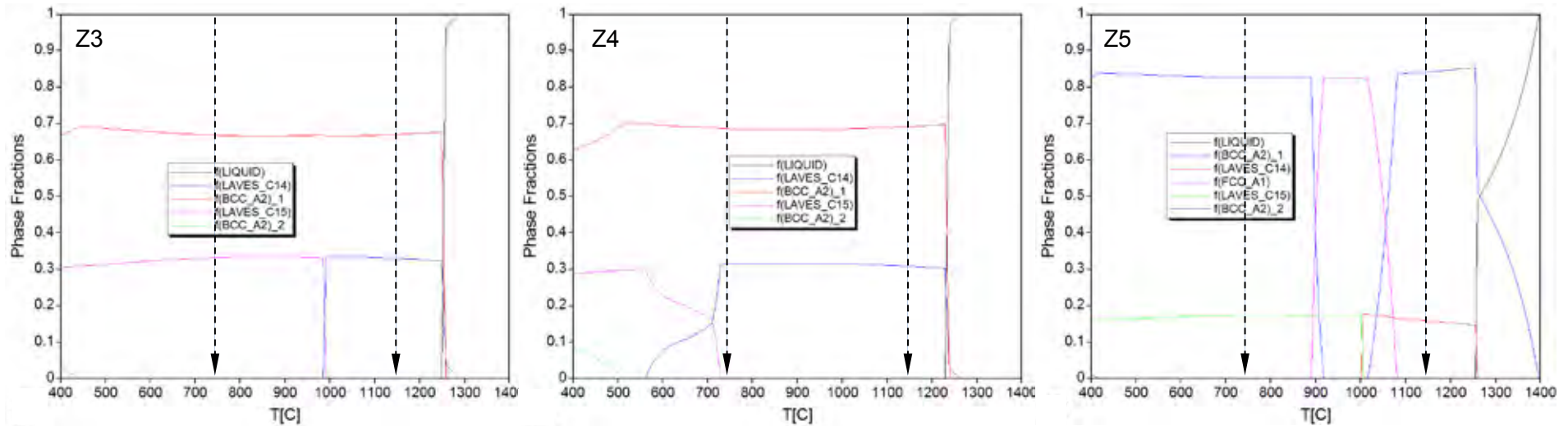
■ Class II: 9Cr ferritic steels

- Better phase stability than 12Cr
- With Grade 91 and 92 as references

■ Class III: High-Cr ferritic steels

- Better corrosion resistance than lower Cr steels, negligible SCC issue
- With AISI 439/444 and 304 as references

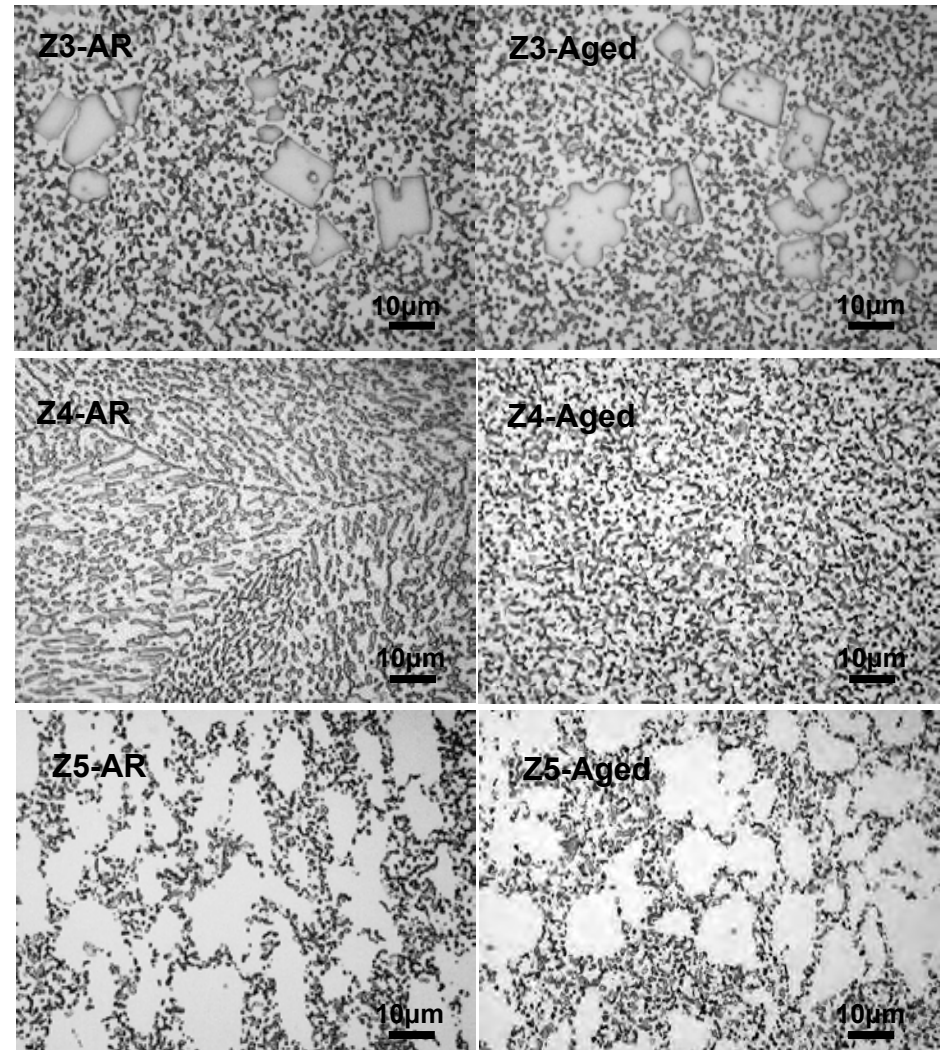
- **Three intermetallics (Fe₂Zr) strengthened alloys Z3, Z4, and Z5 were designed with**
 - Phase transformation of Fe₂Zr from C14-type at high-temperature to C15-type at low-temperature
 - Similar amount of Fe₂Zr in Z3 and Z4, but about ½ amount reduced in Z5.
- **Conditions**
 - As-received: 1150°C + 50% reduction deformation + AC
 - Aging: 750°C for 1800 h to obtain C15-type in Z3 and Z5, comparing its effect to C14-type in Z4.



■ Intermetallics in as-received:

- Z3 is primarily composed of dispersive fine intermetallics with a few large size intermetallics.
- Z4 is dominated by dispersive fine intermetallics with minor orientations (or texture) in different grains.
- Z5 has less amount of fine intermetallics forming a network.

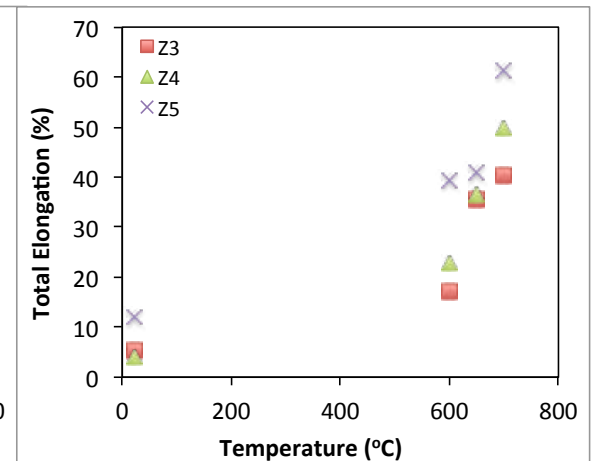
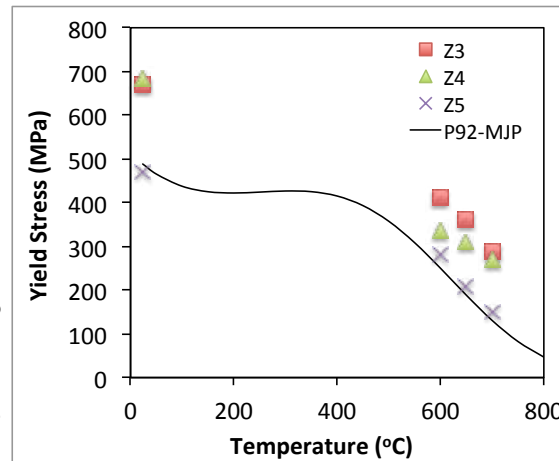
■ The aging did not lead to noticeable changes to the microstructures of Z3 and Z5. But the texture in Z4-AR was eliminated by the aging.



Class I: Intermetallics-Strengthened Alloys

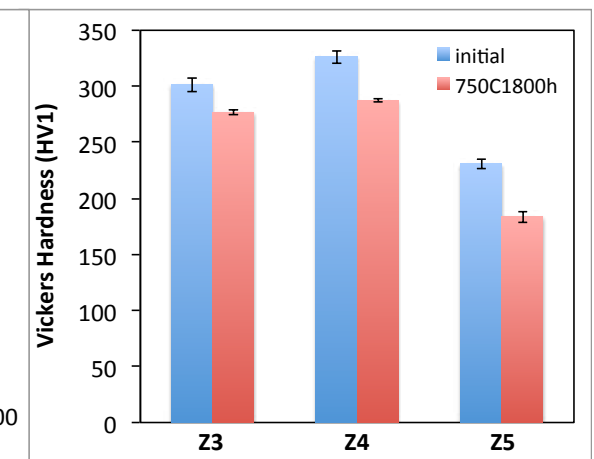
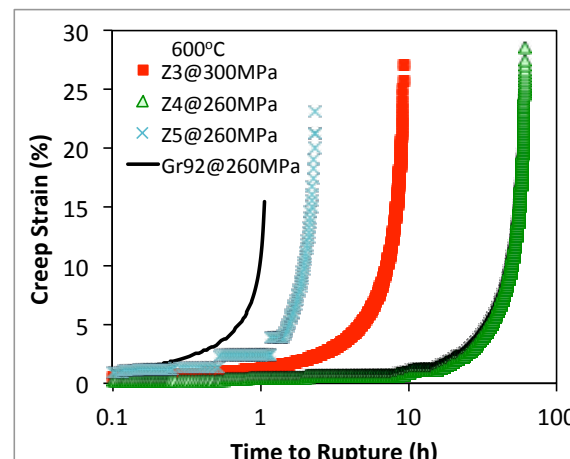
■ Tensile testing results:

- Yield stresses of Z3 are similar to Z4, about 40-60% higher than P92. Yield stress of Z5 is comparable to P92.
- Total elongations of the alloys are comparable or better than P92 at high temperatures, but poor at room temperature.



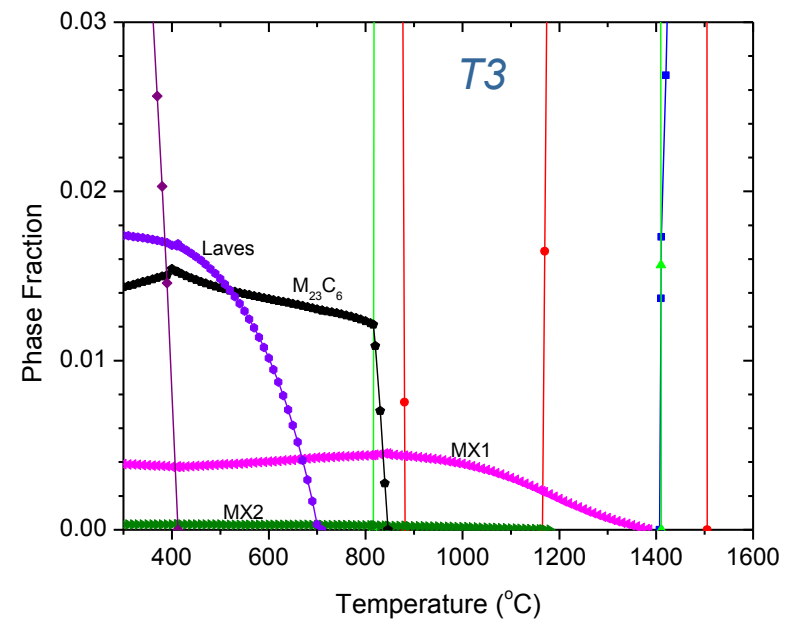
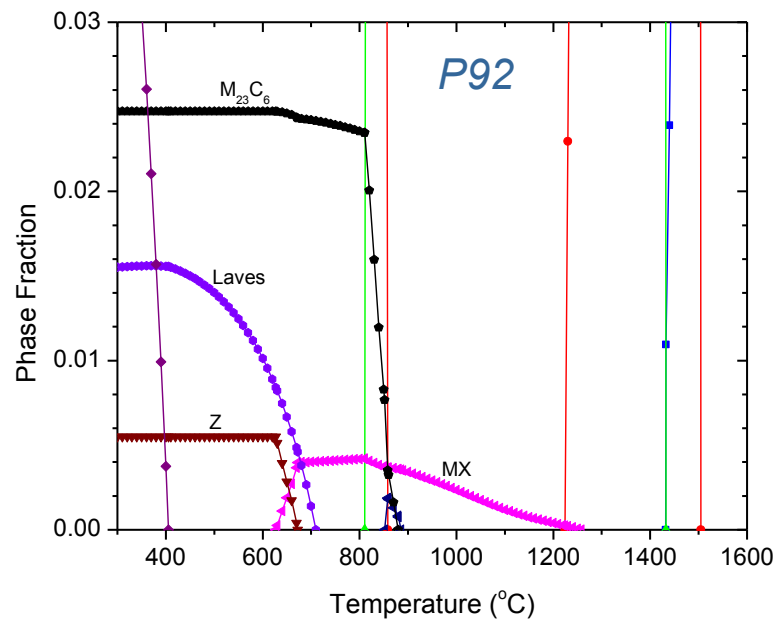
■ Creep testing at 600°C indicated different levels of improvement in creep resistance in both life and ductility.

- The aging slightly reduced the hardness of the alloys.
- The aging may alter the mechanical properties of the alloys, e.g., room temperature ductility.

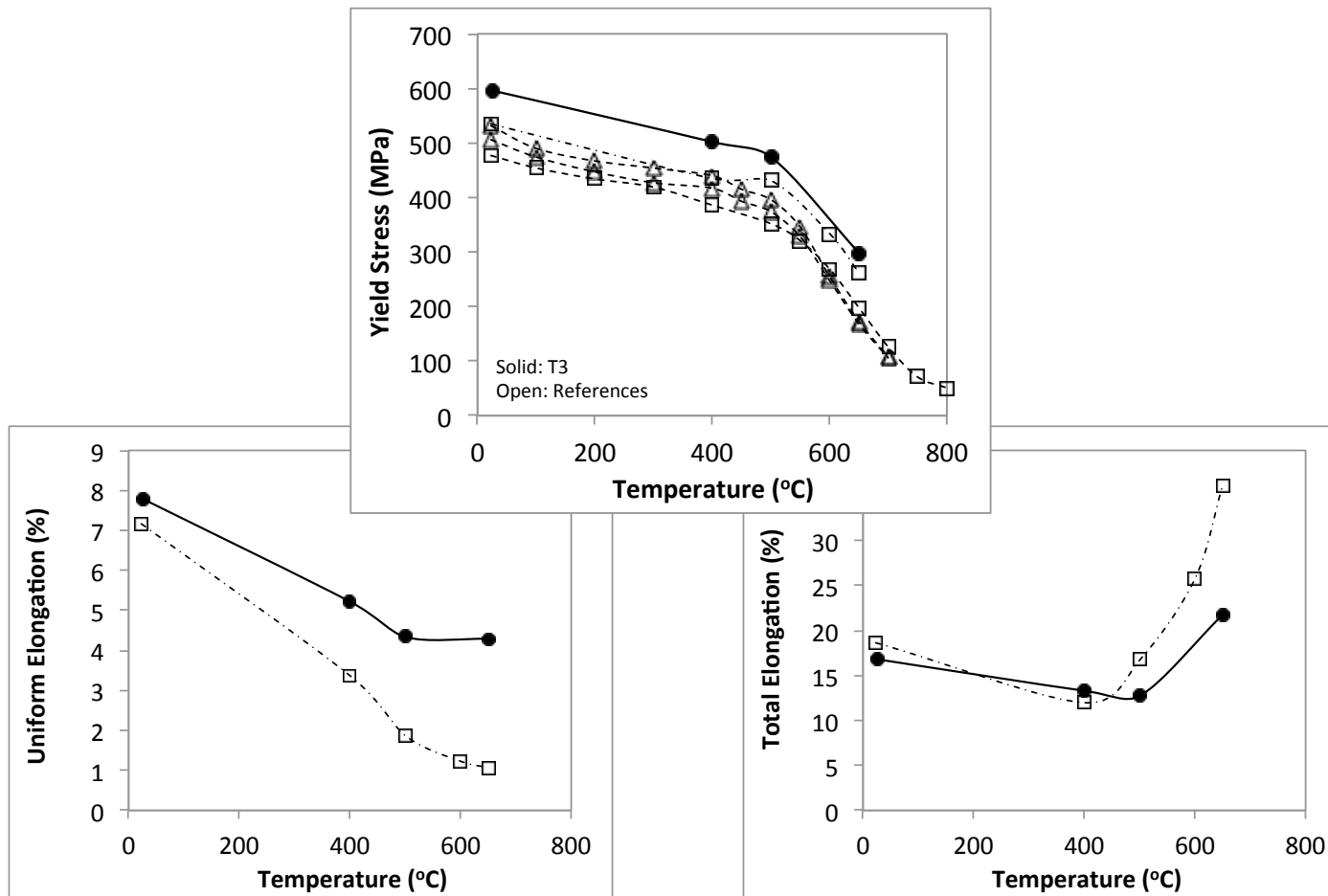


■ Aims:

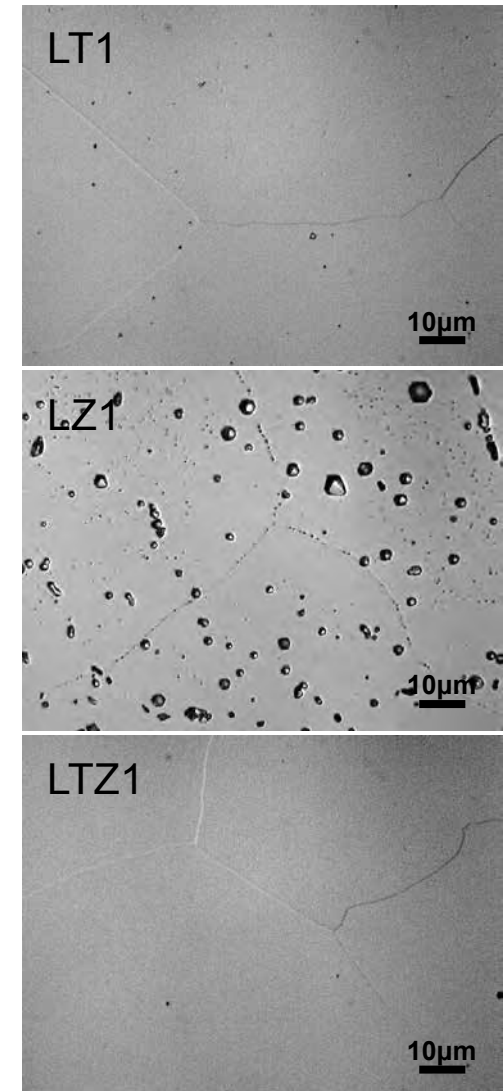
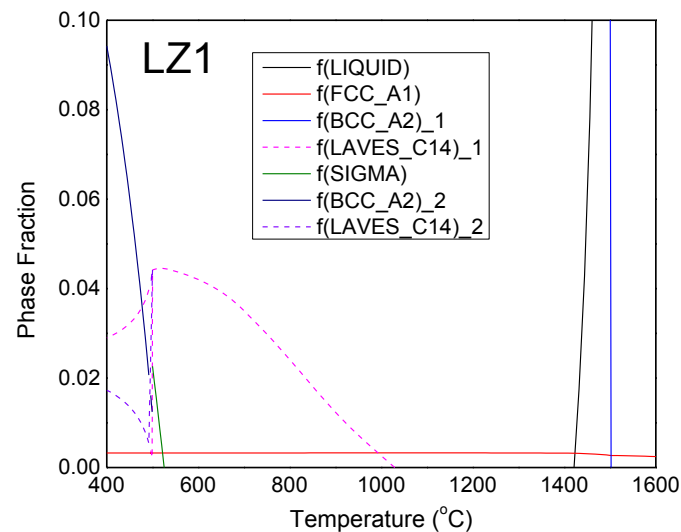
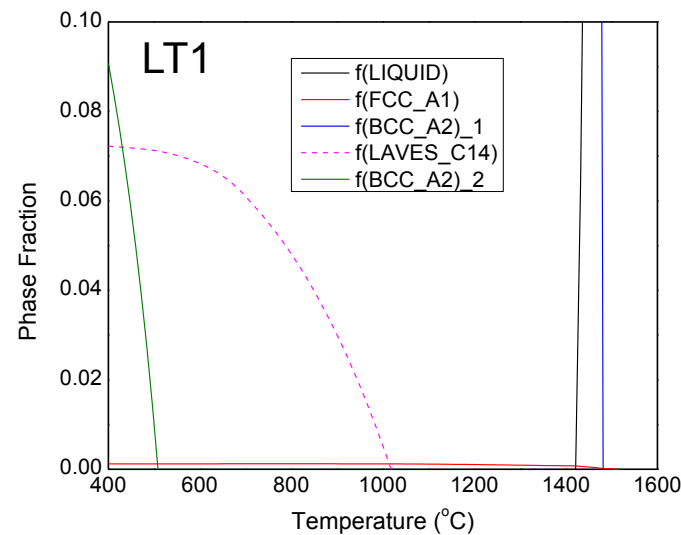
- Eliminate Z-phase and reduce $M_{23}C_6$.
- Changes to Laves phase content is not considered.



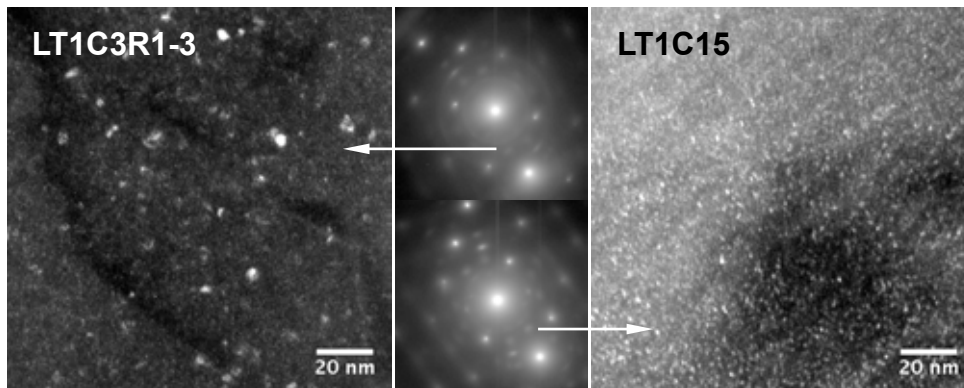
- Tensile testing results of alloy T3 showed moderate increase in yield strength (10-15%) and comparable or slightly lower total elongation, but noticeably improved uniform elongation.



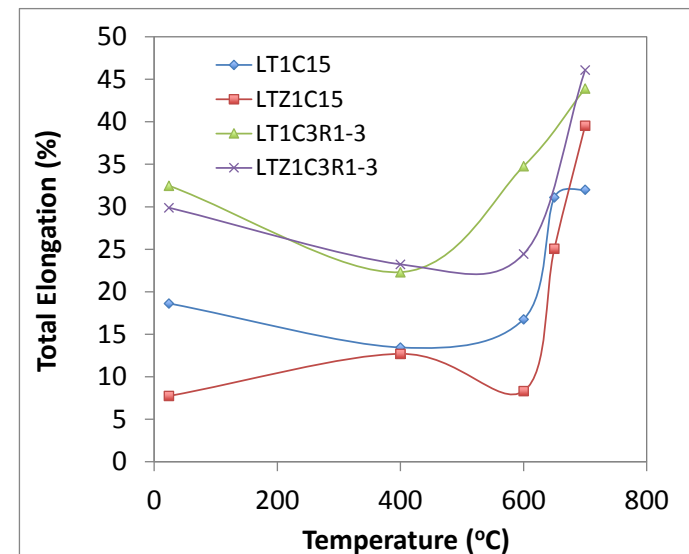
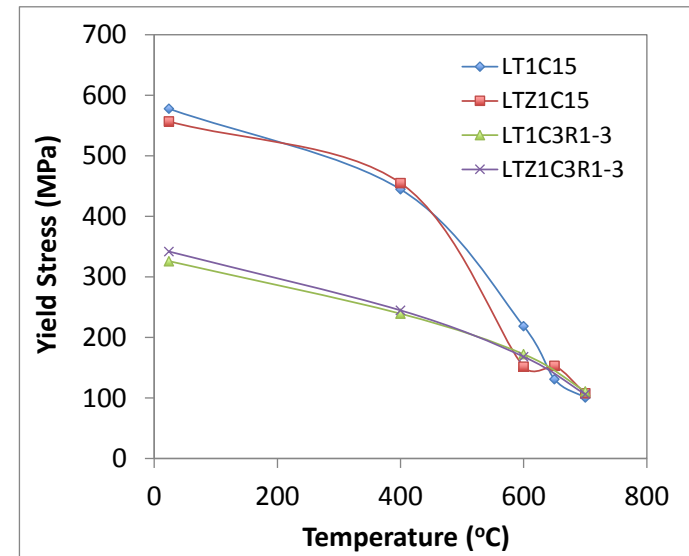
- High-Cr (15Cr) ferritic alloys were designed to have Laves-phase strengthening.
- The poor control of oxygen and nitrogen in LZ1 alloy resulted in a lot of large particles, harmful to mechanical properties.



- **The alloys' strength is adjustable by thermomechanical treatment, which is comparable or superior to AISI 304.**
 - The increased strength would be primarily attributable to the significantly increased amount of ultrafine precipitates.

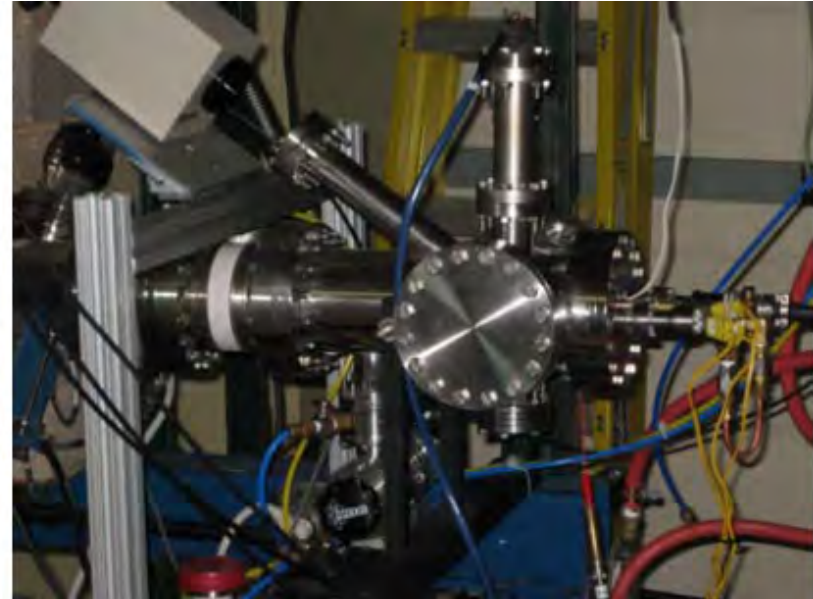


- **The alloys showed decent ductility.**
 - The occasional low ductility was resulted from aggregated Zr-oxide inclusions.



Ion-Irradiation Experiments

- Radiation resistance of the alloys is to be evaluated using proton and heavy ion (e.g., Fe^{2+}) irradiations.
 - Radiation-hardening, radiation-induced phase stability and segregation will be studied.
- Radiation resistance screening of the first set of samples has been initiated.



1.7 MV Tandem Accelerator Ion Beam @ UW-Madison

Zr-alloying would improved mechanical properties and radiation resistance. Approaches to solve the difficulty in alloy chemistry control will be explored by collaborating with commercial vendors, e.g., Carpenter Tech.

FY 2013

- Three classes of alloys are being explored by experiments assisted with computational thermodynamics.
 - Intermetallics-strengthened
 - 9Cr ferritic
 - Higher-Cr (15Cr) ferritic
- Ion irradiation experiments of selected alloy samples are initiated.

FY 2014

- Development and down selection of the 3-class of alloys for larger heats production and mechanical property assessment.
- Ion irradiation study of selected alloy samples.

FY 2015

- Continue testing, ion-irradiation, and characterization of the down-selected alloys.
- Recommend superior alloy(s) for further investigation.