Document Title	Subject Matter	Document Number	Prepared by	Document Source	Date
SRE Fuel Element Damage – An Interim Report	Investigation as to the cause of the power run 14 incident at the SRE, including metallurgical exam of fuel samples and other components of the reactor, chemical analysis of the coolant and its contaminants, radiochemical analyses of the coolant and gaseous activity, investigation of reactivity effects, and examination of mechanisms of damage and potential deleterious effects on the reactor system. Tentative conclusions made based on the investigation.	NAA-SR-4488	The Atomics International Ad Hoc Committee, J.G. Lundholm, Jr., Chairman, et al., Atomics International	www.etec.energy.gov	11.15.1959
SRE Fuel Element Damage – Final Report	Supplements NAA-SR-4488 with conclusions as to the cause of the fuel element failure and modifications made to avoid recurrence of another major fuel element failure	NAA-SR-4488 (suppl)	The Atomics International Ad Hoc Committee, F.L. Fillmore, Chairman, et al., Atomics International	www.etec.energy.gov	1961
Distribution of Fission Product Contamination in the SRE	During the recovery procedures after the fuel element damage in the SRE, collection and analysis of data from this incident was undertaken to support a concurrent project involving experimental investigation into the solubility of certain fission products in sodium.	NAA-SR-6890	R.S. Hart Atomics International	www.etec.energy.gov	3.1.1962
SRE Decommissioning Final Report	Documentation of the decontamination and disposal of the SRE.	ESG-DOE-13403	J.W. Carroll Rockwell International	www.etec.energy.gov	8.15.1983
SRE First Core Fuel - Calculation of Uranium and Plutonium Inventories in the SRE First Core Fuel Loading	Document presents calculation of isotope concentrations in the first core fuel unalloyed uranium, 2.778 weight percent enriched after final exposure. The information was required prior to fuel shipments to ORNL for processing.	TDR No. 5301	J. Kinzer & A.C. Crawford Atomics International	Boeing BNA02335530	5.16.1960
Criticality Study - SRE Cores I and II Fuel	Results presented in the Core I study are repeated and expanded to provide guidance for packaging of multiple containers. Result derived for Core I fuel provide more conservative limits for Core II fuel.	TI-704-990-018 Revision A	R.J. Tuttle Atomics International	Boeing files	1.20.1975
SRE Activity Requirement No. 29, Final Closeout of the SRE Facility.	Plans for final closeout activity of SRE D&D that consisted of a comprehensive rad survey of the entire SRE area, delineating areas where decontamination is incomplete, and resurveying after completion of decontamination.	N704ACR990027	B.F. Ureda Rockwell International	www.etec.energy.gov	10.21.1977
SRE Site Survey Plan for Release to Unrestricted Use.	N704ACR990027 (Doc No. 11)	N704DWP990063	R.J. Tuttle Rockwell International	www.etec.energy.gov	4.7.1978
Final Radiological Inspection of the Below-Grade Areas in the SRE Prior to Release for Unrestricted Use.	Document pertains to activities performed at below-grade areas at the SRE to demonstrate compliance with established criteria. Includes monitoring techniques, data to be collected, analyses to be performed, and backfilling controls.	N704TI990057	John C. Blake Rockwell International	www.etec.energy.gov	8.26.1981
Radiological Survey Plan, Support of D&D Program Operations at T-143 (SRE)	Requirements for rad survey data in support of the D&D of the SRE facility (T-143).	N704TP990008	R.J. Tuttle, et al. Atomics International	www.etec.energy.gov	9.15.1981
Radiological Survey Results – Release to Unrestricted Use, SRE Region I (Building 724 Area).	Results of the rad survey for Region I (Building T724-Hot Oil Sodium Cleaning facility) of the SRE. Results are below the applicable limits and this area may be released to unrestricted use.	N704TI990027	D.E. Owens Rockwell International	www.etec.energy.gov	5.4.1978
Radiological Survey Results – Release to Unrestricted Use, SRE Region II (Building 163, Box Shop).	Results of the rad survey for Region II (Building 163, Box Shop) of the SRE. Results are below the applicable limits and this area may be released to unrestricted use.	N704TI990028	D.E. Owens Rockwell International	www.etec.energy.gov	5.4.1978
Radiological Survey Results – Release to Unrestricted Use, SRE Region III		N704TI990029	J.H. Wallace Rockwell International	www.etec.energy.gov	5.13.1983
Radiological Survey Results – Release to Unrestricted Use, SRE Region IV (West Parking Lot).	use; area adjacent to storage yard (1654) is conditionally released subject to a final soil sample	N704TI990030	D.E. Owens Rockwell International	www.etec.energy.gov	5.4.1978
Radiological Survey Results – Release to Unrestricted Use, SRE Region V (Gas Storage Vault).	Results of the rad survey for Region V (Gas Storage Vault) of the SRE. Results are below the applicable limits and this area may be released to unrestricted use.	N704TI990031	D.E. Owens Rockwell International	www.etec.energy.gov	11.2.1978
Radiólogical Survey Results – Release to Unrestricted Use, SRE Region VI (Water Tank Area).	Results of the rad survey for Region VI (Water Tank Area) of the SRE. Results are below the applicable limits and this area may be released to unrestricted use.	N704TI990032	D.E. Owens Rockwell International	www.etec.energy.gov	11.10.1978
Radiological Survey Results – Release to Unrestricted Use, SRE Region VII	Results of the rad survey for Region VII of the SRE. Results are below the applicable limits and this area may be released to unrestricted use.	N704TI990033	J.H. Wallace Rockwell International	www.etec.energy.gov	5.13.1983
Radiological Survey Results – Release to Unrestricted Use, SRE Region VIII	Results of the rad survey for Region VIII of the SRE. Results are below the applicable limits and this area may be released to unrestricted use.	N704TI990034	J.H. Wallace Rockwell International	www.etec.energy.gov	5.13.1983
Radiological Survey Results – Release to Unrestricted Use, SRE Region IX	Results of the rad survey for Region IX of the SRE. Results are below the applicable limits and	N704TI990035	J.H. Wallace	www.etec.energy.gov	5.31.1983
Radiological Survey Results – Release to Unrestricted Use, SRE Region X	Results of the rad survey for Region X of the SRE. Results are below the applicable limits and this area may be released to unrestricted use.	N704TI990036	J.H. Wallace Rockwell International	www.etec.energy.gov	5.31.1983
Radiological Survey Results – Release to Unrestricted Use, SRE Building 041	Results of the rad survey for Building 041 (SRE component storage) of the SRE. Results are	N704TI990037	J.H. Wallace Rockwell International	www.etec.energy.gov	11.9.1982
Radiological Survey Results – Release to Unrestricted Use, SRE Building 143	Results of the rad survey for Building 143 (major reactor building) of the SRE. Results are below the applicable limits and this area may be released to unrestricted use.	N704TI990038	J.H. Wallace Rockwell International	www.etec.energy.gov	5.31.1983
	Results of the rad survey for Building 163 (CERF) of the SRE. Results are below the applicable	N704TI990039	J.H. Wallace		1
	SRE Fuel Element Damage – An Interim Report SRE Fuel Element Damage – Final Report Distribution of Fission Product Contamination in the SRE SRE Decommissioning Final Report SRE First Core Fuel - Calculation of Uranium and Plutonium Inventories in the SRE First Core Fuel Loading Criticality Study - SRE Cores I and II Fuel SRE Activity Requirement No. 29, Final Closeout of the SRE Facility. SRE Site Survey Plan for Release to Unrestricted Use. Final Radiological Inspection of the Below-Grade Areas in the SRE Prior to Release for Unrestricted Use. Radiological Survey Plan, Support of D&D Program Operations at T-143 (SRE) Radiological Survey Results – Release to Unrestricted Use, SRE Region I (Building 724 Area). Radiological Survey Results – Release to Unrestricted Use, SRE Region III (Building 163, Box Shop). Radiological Survey Results – Release to Unrestricted Use, SRE Region IV (West Parking Lot). Radiological Survey Results – Release to Unrestricted Use, SRE Region IV (Water Tank Area). Radiological Survey Results – Release to Unrestricted Use, SRE Region VI (Water Tank Area). Radiological Survey Results – Release to Unrestricted Use, SRE Region VIII Radiological Survey Results – Release to Unrestricted Use, SRE Region VIII Radiological Survey Results – Release to Unrestricted Use, SRE Region VIII Radiological Survey Results – Release to Unrestricted Use, SRE Region IX Radiological Survey Results – Release to Unrestricted Use, SRE Region IX Radiological Survey Results – Release to Unrestricted Use, SRE Region X Radiological Survey Results – Release to Unrestricted Use, SRE Building 041	SRE Fuel Element Damage – An Interim Report SRE Fuel Element Damage – Finel Report SRE Fuel Element Damage –	heeligiblior a to the cause of the power can 14 incident of the STE. Including metallurgooid coard of this service, commonly making of the coard and leading of the coard of this power commonly making of the coard of the coard of this service, commonly making of the coard of the coard and glaces of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the coard of the fine element failure and process of the fi	heesingsice as in the case of the proof or 114 indexet at de SRE industry mentaluginal expert of the control to search or an either congression of the exchange of the control town control or search or an ellustro congression of the exchange of the control town control or search or an ellustro congression of the exchange of the control town control or search or an ellustro congression of the exchange of the control town control or search or an ellustro congression or an ellustro control or ellustro control or ellustro or ellustro control or ellus	Part Barrier Carego — An Protect Rock Interchalation as to the caregories for the protection of the p

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	Post Remedial Action Survey Report for the SRE Facility, SSFL, Rockwell International	ANL, at request of DOE, conducted a third party independent survey of the SRE. RAD measurements (instrument and smear surveys) and analyses of soil, concrete, asphalt, rock, rebar, water, and sewer sludge samples at SRE were conducted between November 1979 and September 1982 to survey residual rad contamination.	DOE-EV-0005-46	R.A. Wyneen, et al. Argonne National Laboratory	www.etec.energy.gov	February 1984
25	Certification Docket for the SRE and Building 003	Certification from DOE to Rocketdyne for unrestricted use of the SRE complex upon completion of RAD surveys and remedial actions to decontaminate and decommission DOE contaminated areas in the SRE complex and Building 003.		DOE, SFO Operations Office	www.etec.energy.gov	9.24.1985 (?)
26	Approved Sitewide Release Criteria for Remediation of Radiological Facilities at SSFL.	Documents the set of approved guideline values for the release w/o radiological restriction of DOE facilities at SSFL including: annual expected dose; soil, water, and structure surface contamination, and ambient gamma exposure rate. Approved by CA DHS.	N001SRR140131	P.D. Rutherford Boeing	www.etec.energy.gov	2.18.1999
27	Request for Approval to Ship Soil from SRE to a Landfill	Letter requesting approval by CA DHS (at request of DTSC) for shipment of Hg-contaminated soil from the former SRE area		P.D. Rutherford Boeing	www.etec.energy.gov	9.25.2001
28	Sodium Reactor Experiment (SRE) Accident	Short write-up prepared by Boeing regarding the SRE accident including fission gas release and employee and environmental safety.		Boeing	www.etec.energy.gov	3.23.2006
29	Process for the Release of Land and Facilities for (Radiologically) Unrestricted Use	Discussion of release process for facilities that have been utilized for rad operations and/or research, including: rad cleanup standards, characterization survey, decommissioning plans, D&D, waste disposal, reporting, final rad status survey, quality objectives and data quality assessment, verification surveys, dose and risk analysis, etc.		Phil Rutherford Boeing	www.etec.energy.gov	9.17.2007
30	Preliminary RAD Survey of Hg Contaminated Soils East of the Former SRE Building. Survey date: 7/26/2001	Report of rad survey performed by CA DHS for mercury contaminated soils near the former SRE facility. No isotopic concentrations were found above the site wide release criteria.		DHS, Radiologic Health Branch, Radiologic Assessment Unit	www.etec.energy.gov	11.19.2002
31	SRE Decommissioning Environmental Evaluation Report	Evaluation of environmental effects (on air, groundwater, soil, surface soil and introduction of radioactivity to the food chain and contamination of vegetation) that would occur upon release for unrestricted use of the SRE.	ESG-DOE-13367	No preparers indicated Rockwell International	www.etec.energy.gov	2.23.1982
32	SRE Experimental Fuel Program (Interim Report)	Program set up to identify an economical fuel for Na graphite reactors using the SRE. Provides some information regarding design of the SRE.	NAA-SR-3456	B.R. Hayward and J.H. Walter Atomics International	www.rocketdynewatch.org	10.15.1959
33	Evaluation of the Al Nuclear Development Field Lab as Location for Reactor Facilities	Evaluation of the Al Nuclear Development Field Lab with respect to its adequacy as a location for nuclear reactors. Meteorological and hydrological data are provided and a brief description of each reactor facility is presented including info on the program presently being conducted and the future plans for use of each facility. See page V-57 through V-61 for description of SRE.	NAA-SR-7300	R.L. Ashley, et al. Atomics International	www.rocketdynewatch.org	5.25.1962
34	Analysis of SRE Power Excursion of July 13, 1959	Report documenting results of a study to determine the cause of the power excursion during run 14 at the SRE.	NAA-SR-5898	F.L. Fillmore Atomics International	www.rocketdynewatch.org	9.15.1961
35	Final Report - SRE Fuel Decladding	Report summarizing the task of decladding the SRE fuel assemblies, and shipment of the fuel to Savannah River for reprocessing.	N704-FDR-990-004	W.F. Dennison Rockwell International	www.rocketdynewatch.org	3.29.1977
36	Comments of the Boeing Co. on the Reports of the SSFL Advisory Panel - Lochbaum, Beyea, et al.	Boeings comments on all 5 of the reports prepared by consultants hired by the SSFL Advisory Panel. The most relevant document to the SRE is the one prepared by Lochbaum, some information is relevant in the report prepared by Beyea.		Phil Rutherford Boeing	www.etec.energy.gov	11.3.2006
37	SRE Original Accident Release Data (letter to E. Crawford)	Letter from Boeing re: venting of gases from the decay holdup tanks at the SRE. Original accident release data (activities and release rate) for period 7/1/59 to 9/30/59 and previous letter (d. 9/29/2004) to E. Crawford regarding releases, attached.		Phil Rutherford Boeing	www.rocketdynewatch.org	1.21.2005
38	News Release	The original news release on 8.29.1959 regarding the "observation of a parted fuel element."		Atomics International	www.rocketdynewatch.org	8.29.1959
39	Report of the SSFL Advisory Panel	Introductory history of the SSFL and introduction to the five SSFL Advisory Panel "studies".		SSFL Panel	www.ssflpanel.org	October 2006
40	An Assessment of Potential Pathways for Release of Gaseous Radioactivity Following Fuel Damage During Run 14 at the Sodium Reactor Experiment	SSFL Advisory panel's expert's discussion of the release of radioactivity after the SRE incident.		David A. Lochbaum SSFL Panel	www.ssflpanel.org	10.5.2006
41	Feasibility of developing exposure estimates for use in epidemiological studies of radioactive emissions from the Santa Susana Field Laboratory	SSFL Advisory panel's expert's development of exposure estimates as a result of the SRE accident.		Jan Beyea SSFL Panel	www.ssflpanel.org	10.5.2006
42	Procedures for Low-Power Physics Experiment in the SRE	Document presents compilation of procedures for the low-power physics experiments performed with the second core loading of the SRE. On page II-10 there is a handwritten note regarding source strength of the first core.	NAA-SR-MEMO-5337	S.F. Burton, et al. R.W. Woodruff, General Editor Atomics International	ETEC Library Database Report Reference No. LM 00494	8.1.1960
43	Report of John R. Frazier, Ph.D.	Review of and professional opinions regarding SSFL Panel reports prepared by Beyea and Lochbaum		John R. Frazier	www.etec.energy.gov	11.4.2006
44	Review and Evaluation of Report of David A. Lochbaum	Review of and professional opinions regarding SSFL Panel report prepared by Lochbaum		John R. Krsul	www.etec.energy.gov	11.4.2006
45	Construction of the Sodium Reactor Experiment	Brochure detailing the construction (from excavation to completion of the building and construction of the reactor itself) and how the reactor worked. Good overview of the SRE.		Atomics International	Boeing files	No date - but the SCE generating plant is up and running
46	Sodium Reactor Experiment	Power point presentation. Includes photos, local newspaper clippings, construction and operational schedule, engineering design, maps, etc.		Boeing	Boeing files	July 2004

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	SRE Activity Requirement No. 27, D&D of B413 Retention Pond and Sanitary Sewer	Activity Requirements for removal and disposition of radioactive contaminated soil and rock from the retention pond, sanitary sewer system, and related appurtenances exterior to Building 143.	N704ACR990024	A.M. Stelle Rockwell International	DTSC RCRA Investigation Soils Group 6 Historical Documents Web Page Document No. HDMSe00713903	9.14.1981
48	Design Modifications to the SRE During FY 1960	Document details design changes made to the SRE to correct existing functional difficulties or to provide for improved reliability following the July 1959 power run 14 incident.	NAA-SR-5348	G.E. Deegan, et al. Atomics International	www.osti.gov/energycitations/	6.20.1960
49	SRE Standard Operating Procedures	Details operating instructions for the reactor and all systems. Includes forms and SRE group organization. Item K. of Section 1 - General SRE Operating Instructions states that "The operating log book will be kept in the bookcase in the control room. This log will be filled out each shift."	NAA-SR-MEMO-5326	R.E. Durand, et al. Atomics International	www.osti.gov/energycitations/	6.27.1960
50	Statement of Daniel Hirsch, President, Committee to Bridge the Gap	Given before the U.S. Senate Committee on Environment and Public Works, oversight hearing on Cleanup Efforts at Federal Facilities, Washington D.C. September 18, 2008.		Daniel Hirsch Committee to Bridge the Gap	www.committeetobridgethegap.org	9.18.2008
51	Chapter 11, Accidents and Destructive Tests - "What is Past is Prologue" from <u>The</u> <u>Technology of Nuclear Reactor Safety</u> , edited by T.J. Thompson and J.G. Beckerley	This document is referenced in Hirsch's statement (document 50) and is derived from NAA memos and papers published in journals. Prepared under the auspices of the Division of Technical Information, U.S. AEC. Thompson was chair of the AEC's Advisory Committee on Reactor Safeguards and later an AEC Commissioner.		Theos J. Thompson, MIT, Cambridge, MA	Referenced in Document No. 50. CDM's information center located the document.	1964
52	SRE Operating Experience	Published in <i>Nuclear Safety</i> this is reference number 45 in Document 51. If there were any included, the copy does not include references used for this paper.		Robert W. Dickinson Atomics International	Boeing files	March 1960
53	Coolant Block Damages SRE Fuel	Published in the journal <i>Nucleonics</i> the article discusses the sequence of events leading to the damage of the SRE fuel caused by the tetralin leak.	Volume 18, No. 1	Robert W. Dickinson Atomics International	Boeing files	January 1960
54	Airborne Radioactive Contamination in SRE High Bay During Reactor Operation	Memo from R.K. Owen to R.E. Durand advising that the SRE be shut down until sources of airborne radioactive contamination (after power run 14) in the high bay are located and repaired.		R.K. Owen Atomics International	Boeing files	7.17.1959
55	Sodium Reactor Experiment	Tom Rucker (SAIC) added this document. It provides detailed design information about the SRE for the second core loading.	IAEA (International Atomic Energy Agency ?)	Unknown	Tom Rucker, SAIC, obtained this document from a notebook on research reactors that was compiled from a number of sources.	June 1961
56	Shipment of SRE Sodium to Rockwell Hanford Operations	Letter from W.D. Kittinger (AI) to C.D. Jackson (ERDA, Oakland CA) dated June 7, 1977; letter from W.E. Mott (ERDA) to W.D. Kittinger (AI) dated June 17, 1977; and telephone conversation record dated June 27, 1977 between Dr. Kluk (ERDA Headquarters) and G.W. Meyers (AI). All pertaining to shipment of RAD-containing sodium from SRE for disposal at Rockwell Hanford Operations.		Various	Boeing files	June 1977
57	SRE Activity Requirement No. 25. Decontamination and Dismantling of Building 724 and Pad 723	Plans for decontamination and/or removal of Building 724 and the sodium component cleaning Pad 723 so the area can be released for unrestricted use. Building 724 will be removed and contaminated concrete in Pad 723 will be removed, packaged and shipped for burial.	N704ACR990021	B.F. Ureda Rockwell International	DTSC RCRA Investigation - Soils Group 6 Historical Documents Web Page Document No. HDMSE00023442.pdf	3.28.1977
58	Design Modifications to the SRE During FY 1960	Revised version of Document No. 48	NAA-SR-5348 (rev.)	G.E. Deegan, et al. Atomics International	www.osti.gov/energycitations/	2.15.1961
59	SRE Systems and Components Experience - Core II	Provides information as to how the reactor was used after the power run 14 incident and the operation of the SRE Core II. Presented at the Sodium Components Development Program Information Meeting, Palo Alto, CA; August 20-21, 1963	NAA-SR-MEMO-9003	E.N. Pearson Atomics International	www.osti.gov/energycitations/	1963
60	Sodium Reactor Experiment Fuel Irradiation Experience	Evaluation of the unalloyed uranium fuel and fuel element design that made up the first SRE core.	NAA-SR-3887	J.L. Ballif, et al. Atomics International	ETEC Library Database Report Reference No. LM 00514	9.15.1959
61	Sodium Reactor Experiment Operating Experience	Abstract of presentation given at 1961 Winter meeting (November 7-9) of the American Nuclear Society. Published in <i>Transactions of the ANS</i> , Volume 4, Number 2, November 1961.		L.E. Glasgow Atomics International	ETEC Library Database Report Reference No. LM 05180	November 1961
62	Performance of Sodium Reactor Experiment	Paper in <i>Power Apparatus and Systems</i> , Number 42, published by American Institute of Electrical Engineers		J.E. Owens, W.T. Morgan, L.E. Glasgow Atomics International	ETEC Library Database Report Reference No. LM 03286	June 1959
63	Operating Experience with the Sodium Reactor Experiment	Published in Proceedings of the Second UN International Conference on the Peaceful Uses of Atomic Energy (Volume 9, Nuclear Power Plants, Part 2), held in Geneva, Switzerland, September 1 - September 13, 1958		F.E. Faris, L.E. Glasgow, D.H. Johnson, R.W. Campbell, J.E. Owens, and G.E. Deegan Atomics International	ETEC Library Database Report Reference No. LM 04934	September 1958
64	Sodium reactor operating experience	Published in Chemical Engineering Progress , Volume 57, No. 3		R.E. Durand Atomics International	ETEC Library Database Report Reference No. LM 01956	March 1961
65	The Sodium Reactor Experiment	Published in Proceedings of the International Conference on the Peaceful Uses of Atomic Energy (Volume 3, Power Reactors), held in Geneva, Switzerland, August 8 - August 20, 1955. General information about the design and operation of the SRE.		W.E. Parkins North American Aviation, Inc.	ETEC Library Database Report Reference No. LM 05527	August 1955
66	Sodium Reactor Experiment	Published in <i>Proceedings of Symposium on Sodium Reactors Technology</i> , Lincoln, NE, May 24-25, 1961. Sponsored by AEC and Consumers Public Power District. General information about the design and operation of the SRE and discussion of the tetralin leak.	TID-7623	L.E. Glasgow Atomics International	ETEC Library Database Report Reference No. LM 01716	May 1961
67	Sodium Graphite Reactors	Presented at the Second International Conference on the Peaceful Uses of Atomic Energy, Geneva, Switzerland. 255 pages of the 278 book are devoted to the SRE. Extensive treatise on the design and operation of the SRE.		C. Starr and R.W. Dickinson Atomics International	ETEC Library Database Report Reference No. LM 01478	September 1958

CDM Doc.	Document Title	Subject Matter	Document Number	Prepared by	Document Source	Date
No.	Document ritie	Subject Matter	bocument Number	Trepared by	bocument source	Date
68	Proceedings of the SRE-OMRE Forum	Four papers on the SRE included in this volume. The forum was held in Los Angeles, CA, February 12 and 13, 1958	TID-7553 (NAA-SR-2600)	L.E. Glasgow, A.C. Werden, Jr.; R.W. Dickinson, R.L. Carter Atomics International	ETEC Library Database Report Reference No. LM 00185	May 1958
69	Decommissioning of the SRE	Abstract of presentation given at 1977 Annual meeting (June 12-16) of the American Nuclear Society. Published in <i>Transactions of the ANS</i> , Volume 26.		W.F. Heine, A.W. Graves, B.F. Ureda Atomics International	ETEC Library Database Report Reference No. LR 05497	June 1977
70	Progress Report on Decommissioning of the SRE	Abstract of presentation given at 1978 Winter meeting (November 12-16) of the American Nuclear Society. Published in <i>Transactions of the ANS</i> , Volume 30.		G.W. Meyers, W.D. Kittinger Atomics International	ETEC Library Database Report Reference No. LR 08603	November 1978
71	Design and Development of Components for the SRE	Presented at the Nuclear Engineering and Science Congress, December 12-16, 1955 in Cleveland, Ohio. Published by American Institute of Chemical Engineers.		W.E. Parkins North American Aviation, Inc.	ETEC Library Database Report Reference No. LM 02837	December 1955
72	Proceedings of the SRE-OMRE Forum	Session I includes 9 "papers" regarding different aspects of the SRE	TID-7525 (NAA-SR-1804)	W.E. Parkins, D.N. Peterson, J.C. Cochran, F.E. Faris, A.C. Werden, Jr., D.T. Eggen, C.C. Woolsey, R.L. Carter, and R.C. Gerber Atomics International	ETEC Library Database Report Reference No. LM 00976	November 1956
73	Retirement of the Sodium Reactor Experiment	Post operational plans are described, plans for retirement of the SRE along with alternate plans that were considered are described. Current status of plant is described and means presented for maintenance of the plant in its present conditions.	AI-AEC-12572	SRE Staff, R.R. Eggleston, editor Atomics International	ETEC Library Database Report Reference No. LM 06461	8.30.1968
74	Facilities Dismantling Plan for SRE	Plan for further dismantling and decon of SRE, including removal of sodium systems, passivation of residual sodium, storage of primary sodium, scrapping or burial of components, removal of reactor vessels. Facilities will be decontaminated to allow for unrestricted use.	FDP-704-990-003	W.F. Heine and B.F. Ureda Rockwell International	Boeing files	6.26.1975
75	Construction of the SRE Core	Published in Volume 13, No. 6 of <i>Nucleonics</i> . Describes arrangement of fuel, graphite, sodium flow and core enclosures as background for understanding methods used to make the zirconium-canned moderator-reflector units.		R.L. Olson North American Aviation, Inc.	ETEC Library Database Report Reference No. LM 02345	June 1955
76	Preliminary Safety Evaluation of the Sodium Reactor Experiment	A study of the possible hazards associated with the operation of the proposed SRE at SSFL.	NAA-SR-Memo-902-Rev	W.E. Abbott, editor and staff North American Aviation, Inc.	ETEC Library Database Report Reference No. LM 03281	2.19.1954
76a	Preliminary Safety Evaluation of the Sodium Reactor Experiment	This is a version of Doc. No. 76 in which someone has blacked out references to "sabotage" in Section II Conclusions, and made some handwritten edits on pages 13, 14, 21, 22, and 25. A community stakeholder provided this version.	NAA-SR-Memo-902(Rev.(Del.))	W.E. Abbott, editor and staff North American Aviation, Inc.	Community Stakeholder	2.19.1954
77	Flux Distributions and Leakage Currents for SRE, P-16	Criticality calculations made for two different diameter tanks with the output being leakage out of the core. To be used in shielding calculations.	NAA-SR-Memo-865 Rev.	R. Balent Atomics International	www.osti.gov/energycitations/	1.29.1954
78	Some Nuclear Experiments on the SRE	Experiments included: dry subcritical assembly, subcritical multiplication measurements to determine the worth of various types of elements, flux mapping of the subcritical reactor, and wet critical assembly.	NAA-SR-MEMO-1956	C. Guderjahn Atomics International	www.osti.gov/energycitations/	6.5.1957
79	Mercury Release at SRE	Conversation between Art Lenox, Boeing and Jack McKernin, Boeing regarding spill of about 1 pint of mercury during demolition of the SRE steam generator		Art Lenox, Boeing	DTSC RCRA Investigation - Soils Group 6 Historical Documents Web Page Document No. HDMSE00025532.pdf	March 2000
80	SRE Core Recovery Program	Discussion of the recovery of the SRE core after the July 1959 incident. The SRE Core Recovery Program assessed the extent of the reactor damage, removed the damaged fuel element components and other material from the reactor core, and replaced defective moderator cans with the objective of returning the SRE reactor core back to a condition that would allow continuation of research and development operations using new loadings of reactor fuel.	NAA-SR-6359	W.J. Freede, Atomics International	ETEC Library Database Report Reference No. LM 00887	12.1.1961
81	Safety Evaluation of Sodium Graphite Reactors	Discussion of the safety evaluation of the type of sodium-graphite reactors in the design and construction stages at Atomics International, including the SRE.	NAA-SR-1626	R.C. Gerber Atomics International	ETEC Library Database Report Reference No. LM 00493	5.1.1956
82	Metallurgical Aspects of SRE Fuel Element Damage Episode	Discussion of metallurgical aspects of the melted SRE fuel elements. Both damaged and undamaged elements were examined.	NAA-SR-4514	J.L. Ballif Atomics International	ETEC Library Database Report Reference No. LM 00279	10.15.1961

AEC = Atomic Energy Commission

AI = Atomics International

CERF = Component Equipment Repair Facility

D&D = decontamination and disposition

ERDA = Energy Research and Development Administration

OMRE = Organic moderated reactor experiment

SCE = Southern California Edison

Textbook

Available from National Technical Information Service (NTIS)

Conference proceedings or journal papers