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## FY17 ART I&C Program Overview and Objectives

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# Where Does ART I&C Program Fit Within DOE-NE Reactor ICHMI Portfolio?

- DOE-NE's reactor I&C and humanmachine interface technology has three primary branches
  - Advanced reactor technologies
    - Fast reactors
    - High temperature gas reactors
    - Molten salt reactors
    - Energy conversion
  - Nuclear energy enabling technologies
    - Advanced sensors and instrumentation
    - Spans reactors and fuel cycle technologies
    - Cross-cutting R&D
  - Light water reactor sustainability
    - Small modular reactors
    - Improved reliability, sustain safety, and extend the of LWRs











## Why Have an Advanced Reactor I&C Program?

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#### Improved I&C is Necessary for Acceptable Performance

Enabling lower operating costs and higher plant reliability

Older I&C technologies remain viable

Multiple advanced reactors were built and operated decades ago Advanced power reactors are not currently operating in market economies

Advanced reactors have had substantial I&C related performance issues



# Why Have I&C As a Cross-Cutting Program?

- Many I&C issues are common to any advanced reactor
  - More efficiently resolved by employing a common solution
- I&C is an enabling technology consequently its issues are frequently not perceived to be the most pressing for any one reactor class
  - Pooling needs enables I&C to receive adequate early attention
- Regulatory requirements and industry standards are based upon common nuclear power safety issues not reactor class
- 1. Developing common technologies to support automating maintenance and inspection activities within containment environments
- 2. Developing a common approach to instrumentation classification in highpassive safety reactors
- 3. Developing common technologies to provide optical access across containment boundaries
- 4. Developing a common approach to post-accident monitoring for reactors without grid access requirements



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### Emphasis of I&C Will Shift Towards Enabling Improved Performance In Future Nuclear Plants



## More detailed condition and process knowledge

- Increasing automation increased availability, lower staffing costs, improved grid transient response
- Maximizing component life
- Enabling improved component performance
- Support for safeguards

#### Increased passive safety

- Larger operating margins & slower accident progression facilitates use of complex digital logic
- Passive shutdown and passive decay heat rejection decrease safety significance of I&C (except for monitoring)



## Deeply Integrated I&C Will Be a Key Difference Between Past and Future Nuclear Systems

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- Sensors and controls have not typically been embedded in nuclear power reactor components
  - High speed simulation and signal processing was not available in the first nuclear era
  - Dense sensor interconnection expands the set of degradation mechanisms that can be observed
- Embedded I&C enables faster control reaction and increased stability in the event of component failures compared with traditional control
  - Traditional approach to large component design is to include mass, large margins, and tolerate inefficiency as cost of doing business
- Makes inherently unstable configurations stable ⇒ smaller, lower mass, lower cost, more reliable
  - Railroad—AC traction drive locomotives enables 50% thrust increase
  - Industrial tools—Sawstop® prevents saw blade amputations
  - Aircraft/Aerospace—stabilizing fundamentally unstable wing configuration
  - Modern jet engines have experienced a 1000X reliability improvement with embedded I&C



Locomotive AC Traction Motors





**Delta Wing Aircraft** 



### Instrumentation is Central to Monitoring Fissile Material Location for Safeguards

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# Nuclear material must be accounted for at each stage of operations

 Material balance measurements and key measurement points are central to safeguards

### Some advanced reactors embed more of the fuel cycle into reactor facility

- Integral fast reactor
- Molten salt reactors
  - Current safeguards implementations do not address implications of fluid fuel forms

### Additional monitoring likely to be required that doesn't exist today

Item counting and visual accountability of fuel may not be possible

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Safeguards are the technical means for the IAEA to verify that States are meeting their legally binding undertaking not to use nuclear material or other items for illicit purposes



# Improved I&C is a Key Element of Achieving Program Missions





# **I&C Projects Also Support NE and ART Initiatives**





Johnson Noise Thermometry for Drift-Free Temperature Measurements

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#### Relevancy

- Instrumentation naturally drifts from calibration over time and each channel must be periodically visited, inspected, and recalibrated.
  - Any potential drift in the primary loop temperature measurement must be accounted for in the operating margin. Hence, decreasing the temperature measurement uncertainty can directly increase the plant revenue.



#### Technical Approach/Accomplishments/Results Expected Deliverable & Schedule

- New Johnson Noise Thermometry system developed
- All hardware and software is complete
- Tested electromagnetic interference (EMI) effects on the system
- Tested at Sandia and at Kingston Steam Plant

- Project is complete
- ORNL/TM-2016/301 provides project details



## **Under Sodium Viewing - ANL**

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#### Developing an enabling under sodium viewing (USV) technique for nondestructive examination (NDE) of SFRs:

- -Real-time operation or maintenance monitoring of SFR at high temperatures and high radiation in-sodium
- -In-service inspection and repair of components, structures, and systems within reactor core or steam generators

#### Technical Approach/Accomplishments/Results

- Constructed a USV test facility for automated in-sodium test, signal/image processing, and defection detection.
- Successfully demonstrated ultrasonic waveguide transduce (UWT) technique with real-time detection resolutions of 0.5 mm in both width and depth up to 343°C in sodium.
- Successfully demonstrated submergible high-temperature transducers with real-time detection resolutions of 0.5 mm in depth and 1 mm in width up to 343°C in sodium.
- Developed a brush-type ultrasonic waveguide transducer phased array (BUWT-PA). Water mockup has shown realtime detection resolutions of 0.5 mm in depth and 0.5-2 mm in width (scanning methods). **ART I&C Overview**



- Development of submergible HT transducers
- Development of BUWT phased array
- In-sodium test of submergible HT transducers and BUWT phased array
- Identify commercial partners and in-reactor USV system integration pathways



### **Under-Sodium Viewing - PNNL**

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#### Ultrasonic Testing for Under Sodium Viewing to:

- Monitor operations in optically opaque sodium at high temperatures
- -Nondestructively inspect structures, systems, and components within the reactor.



#### Wrap-up and freeze PA-UT probe designs

- Develop formalized design documentation
  - Linear and matrix-array probe designs
- Develop technology transfer materials
- Complete IP protection (patent submission) and export control reviews
- Identify and engage potential commercial partners
  - Complete nondisclosure agreements and hold technical discussions
- Complete image reconstruction/signal processing study for improving signal-to-noise ratio, imagebased detection and characterization in-sodium

- Complete export control analysis (Near Completion)
- Protect IP, submit patent (In-Progress)
- Engage viable commercial partners (In-Progress)



Measurement Technologies for Prognostic Indicators for Advanced Reactor Passive Components

- Enhanced awareness of AdvRx component condition to improve asset protection and lifetime degradation management, optimized O&M activities, extended operating cycles, and extended reactor life
  - Detection of early stages of degradation in inaccessible or hard-toreplace passive components key to safe operation of AdvRx
  - Identification of measurement technologies that provide early indicators of damage in passive components and reactor internals
  - Evaluation of in-situ measurement sensitivity and effectiveness



- FY2017 focus on sensor design, fabrication, and assessment of inspection capability for in-situ online monitoring of selected degradation modes
  - Technical approach uses a combination of simulation modeling and experimental assessment of selected in-situ nondestructive measurements for their sensitivity and reliability
- Accomplishments/Results:
  - Target components and degradation mode identified; experimental plan draft completed
  - Insights into likely in-situ measurement options in AdvRx
  - Probe designs being finalized and fabrication initiated
  - Ongoing/planned efforts focus on completing probe fabrication for high-temperature in-situ monitoring, and quantification of performance

- Complete initial evaluation of in-situ measurement of AdvRx passive component prognostic indicators
- Complete sensor design and fabrication
- Complete initial experimental assessment of in-situ monitoring options, with a focus on sensitivity and measurement uncertainty



### **High Temperature Fission Chamber**

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Objective – Design, build, and demonstrate a high temperature fission chamber

#### HTFC Fabrication

- Irradiation test plan and transportation plan under completed
- 100% of parts received
- High temperature preliminary experiment at ORNL completed
- Activation analysis of detector and test assembly completed

#### HTFC Electronics

- Noise analysis of furnace power supply completed
- DAQ ordered and control software started



**Temperature Experiment** 



Instrumentation Will Continue to Form the Nervous System of Future Nuclear Power Plants

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## Instrumentation remains an enabling technology

- Traditional instrumentation technologies remain viable
- Lowering costs and increasing reliability requires improved I&C
- Integrating I&C into the plant design is key to effective operations & maintenance as well as safeguards
- Diagnostics & prognostics can facilitate limited lifetime components

